

Research
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Progress on Assessment of NSTX Divertor Particle and Heat Fluxes

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ALPS/APEX April Meeting
Grand Canyon, CO
April 7-11, 2003



NSTX Fueling Observations

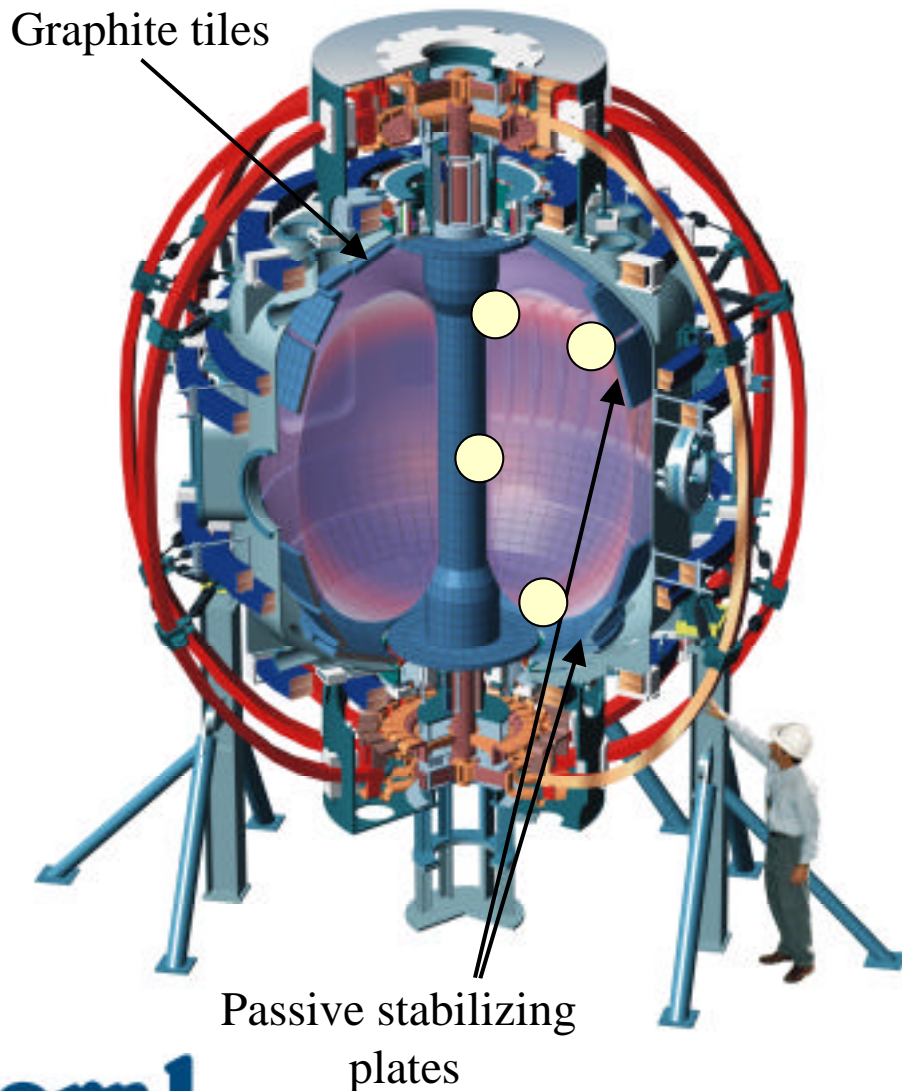


- NSTX H-modes suffer uncontrolled density rise
 - Due to limited fueling control from center stack injector?
- NSTX L-modes also exhibit density rise
 - No center stack injector used

-> NSTX generally has high recycling

- Recycling control needed, but where are particles and power?

NSTX Explores Low Aspect Ratio ($A=R/a$) physics regime



Parameters	Design	Achieved
Major Radius	0.85m	A 1.27
Minor Radius	0.67m	
Plasma Current	1MA	1.5MA
Toroidal Field	0.6T	0.6T
Heating and Current Drive		
NBI (100keV)	5MW	7 MW
RF (30MHz)	6MW	6 MW

Wall Conditioning:

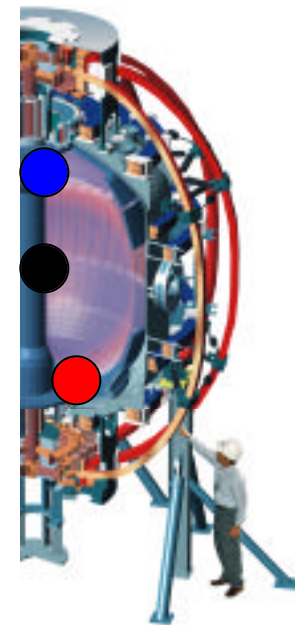
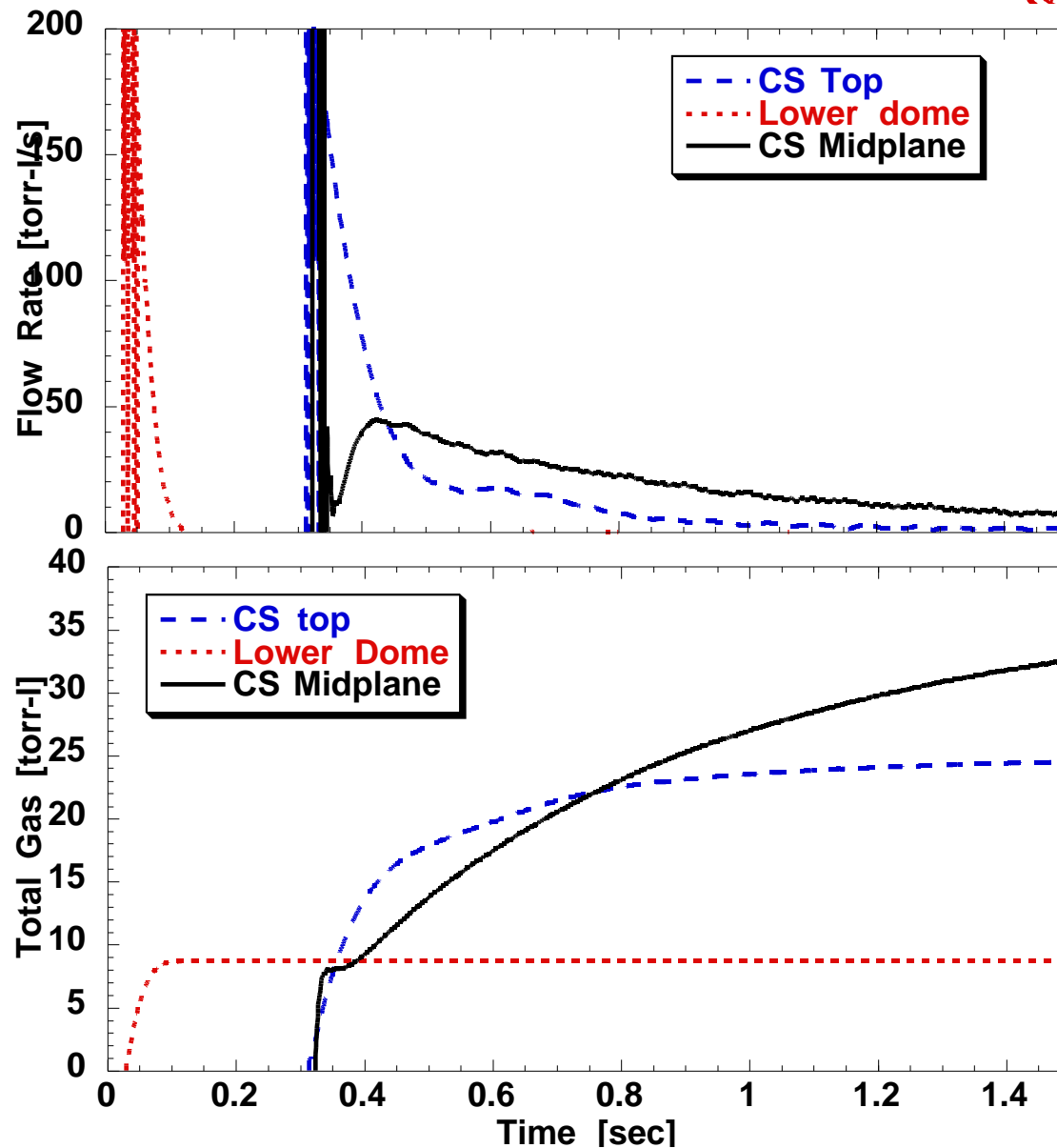
350 deg. bakeout of graphite tiles

Regular boronization (~3 weeks)

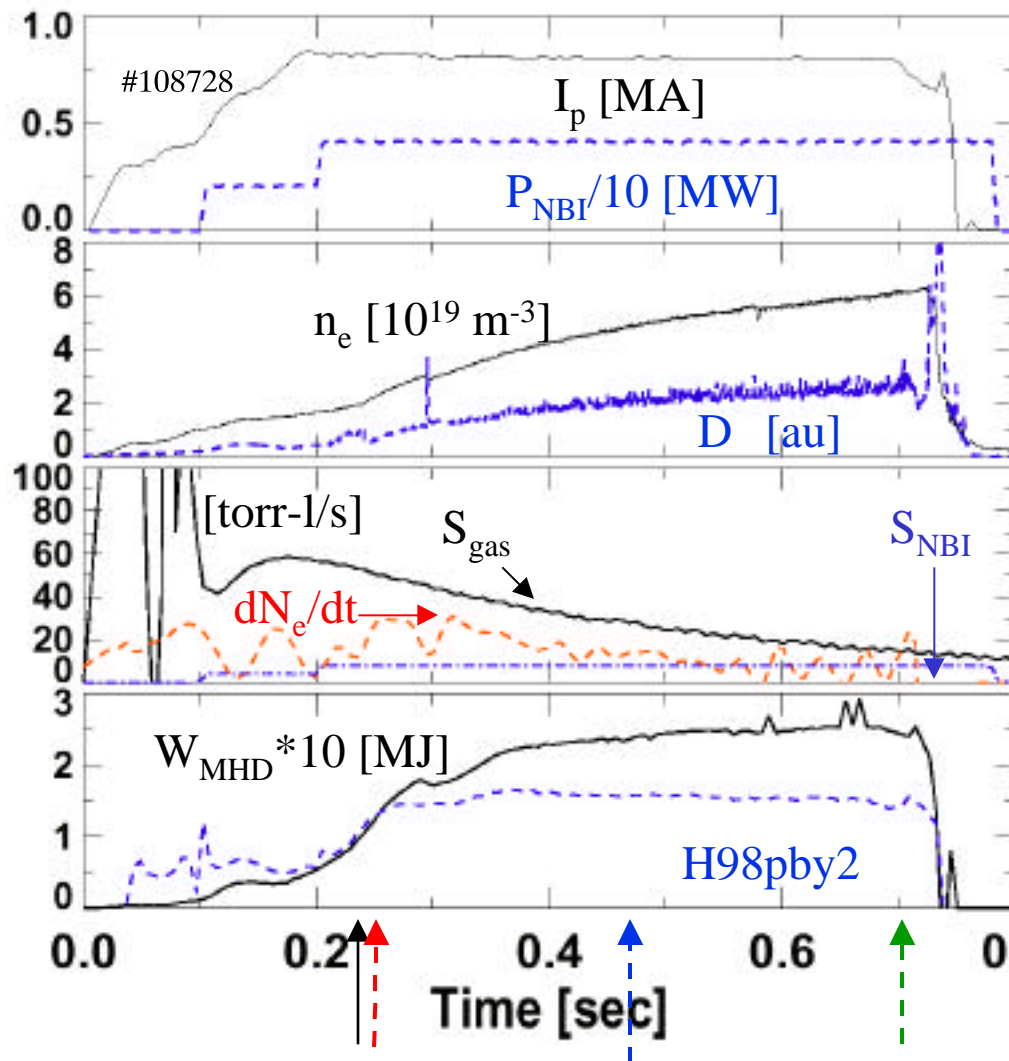
Helium Glow between discharges

Center stack gas injection

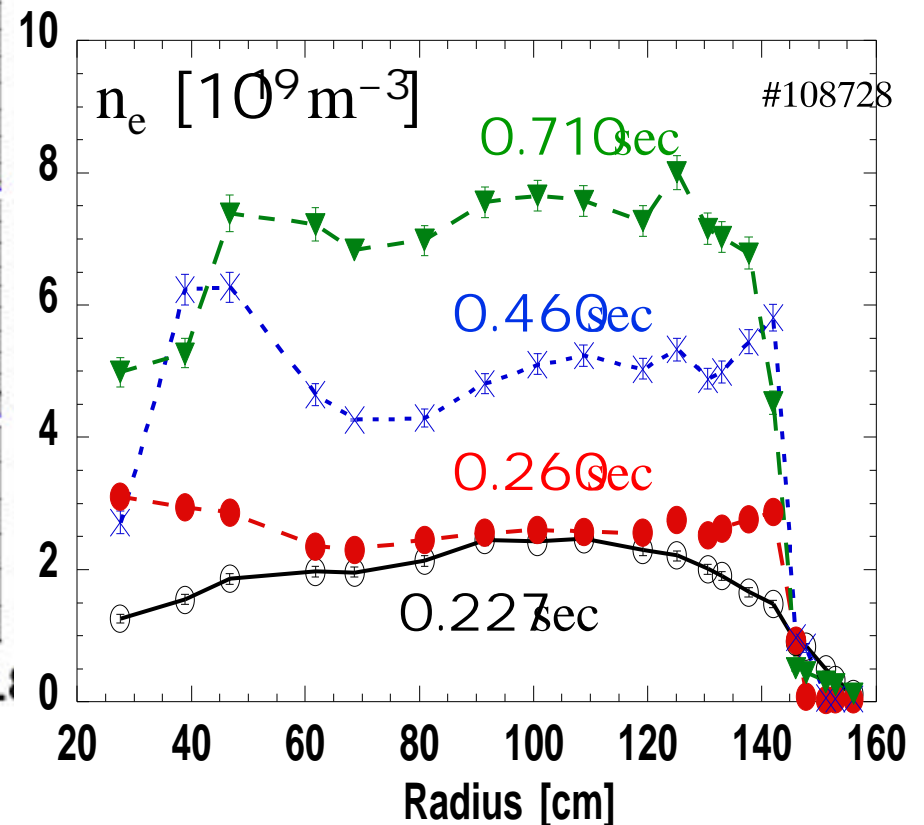
Load-and-Dump Gas Injectors Have Different Flow Characteristics and Delay Time



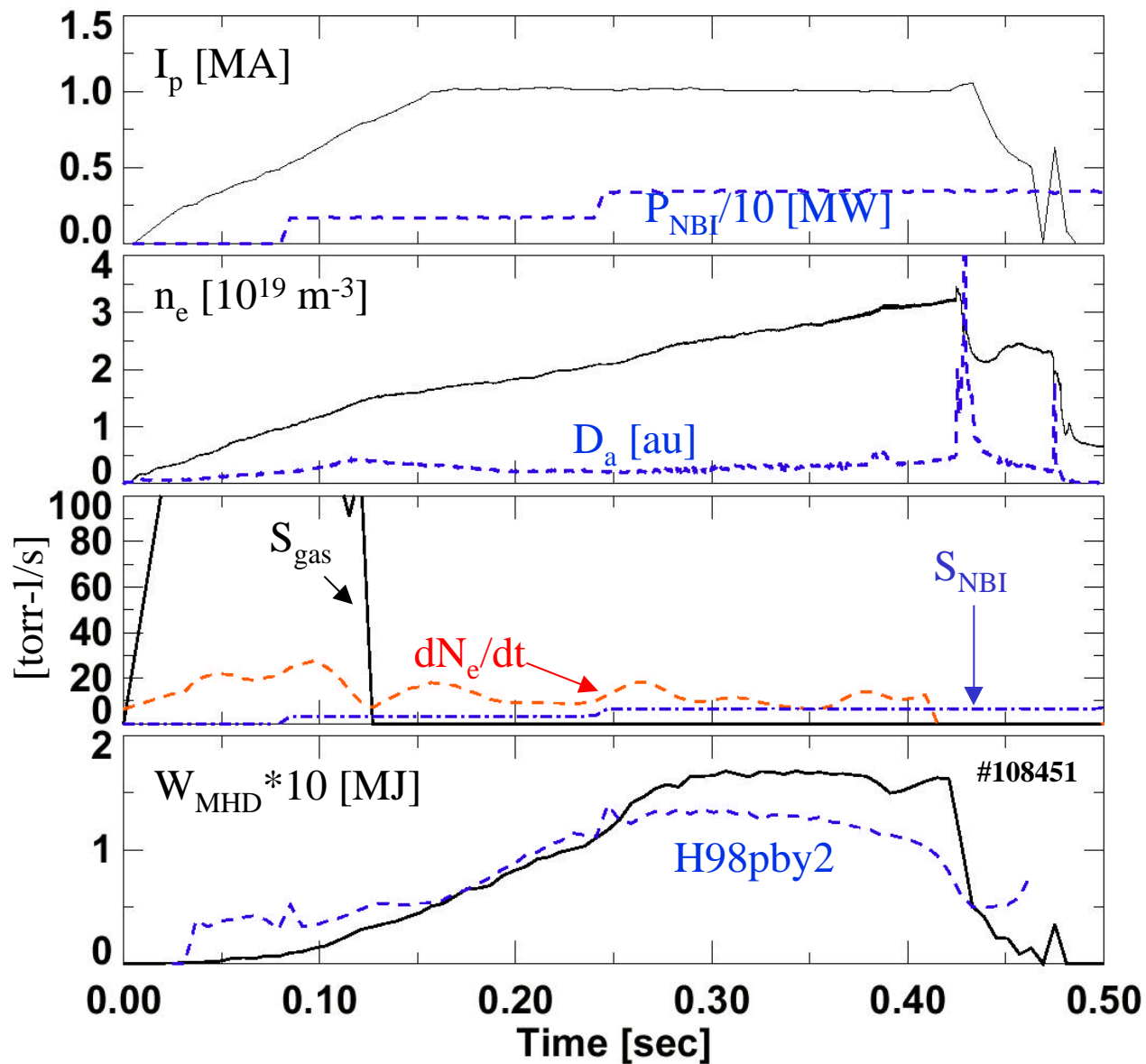
Uncontrolled (non-disruptive) density rise in long pulse H-modes



- Density control needed for improved current drive efficiency, transport studies, and power and particle handling research



Density also rises faster than NBI fuel rate in long pulse L-modes



Classic Particle Balance Model used to Estimate Particle Containment Time



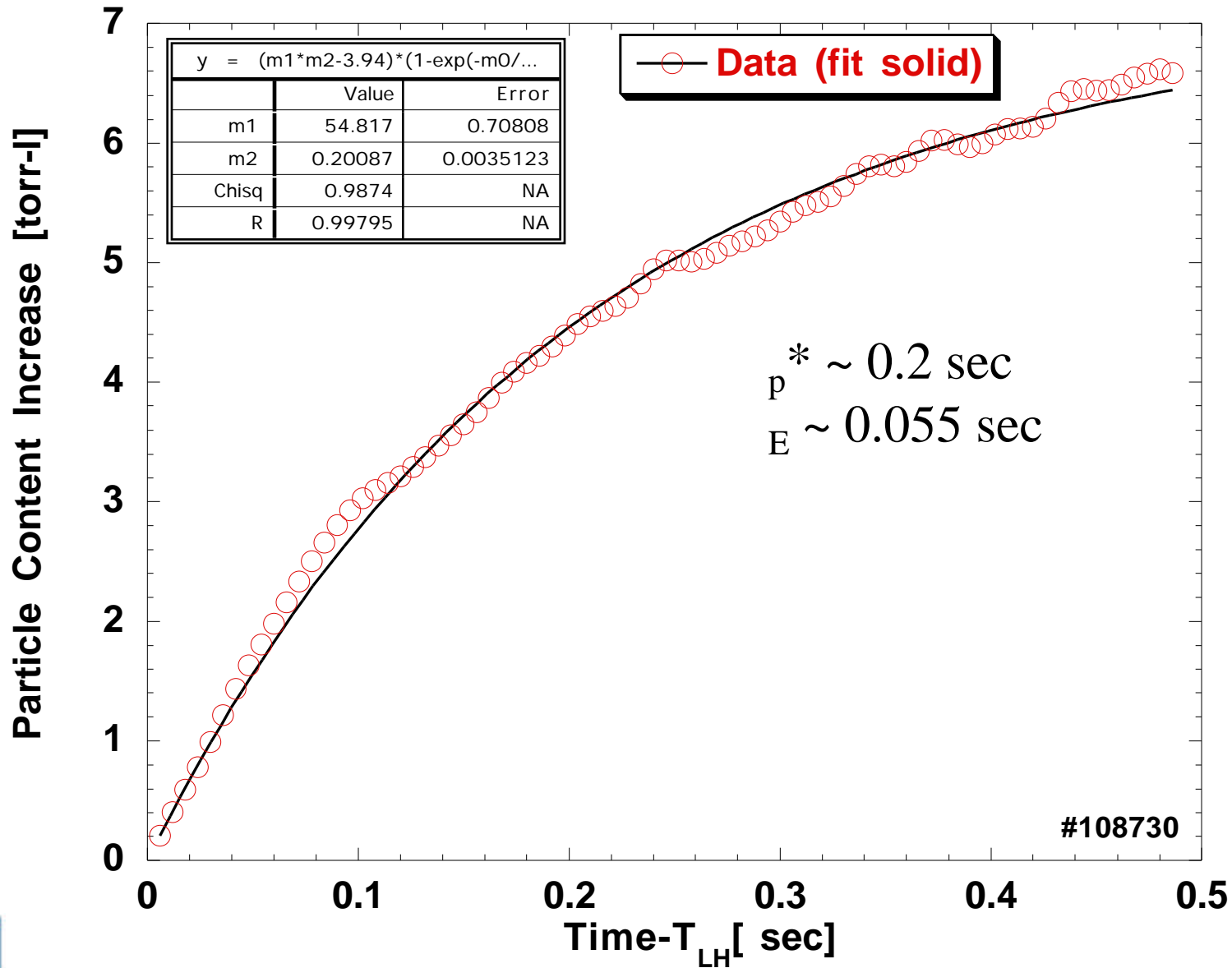
- Assuming constant NBI and gas source term, time dependence of plasma content has analytic solution

$$\frac{dN}{dt} = \eta_{NBI} S_{NBI} + \eta_{gas} S_{gas} - \frac{N}{\tau_p^*}$$

$$\text{where } \tau_p^* = \frac{\tau_p}{1 - R_{eff}}$$

$$N(t) = N_o \exp \left(-\frac{t}{\tau_p^*} \right) + \tau_p^* (\eta_{NBI} S_{NBI} + \eta_{gas} S_{gas}) \left(1 - \exp \left(-\frac{t}{\tau_p^*} \right) \right)$$

Particle Containment Times ~ 0.2-0.3 sec. in NSTX with constant gas fueling rate model



Classic Density Rise Model Can Be Modified with Realistic Time Dependencies



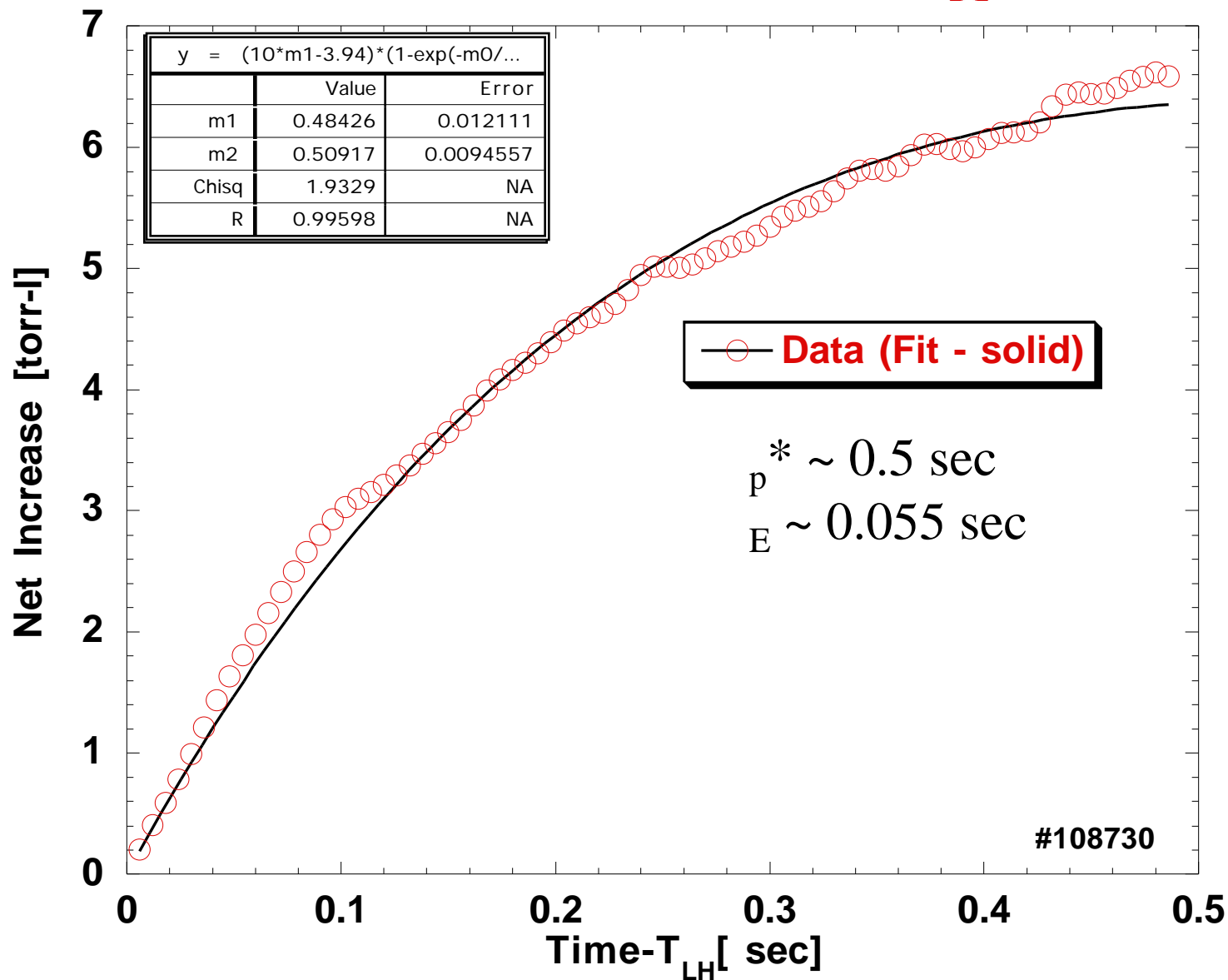
- Assuming time dependent gas source term; time dependence of plasma content has analytic solution

$$\frac{dN}{dt} = \eta_{NBI} S_{NBI} + \eta_{gas} S_{gas}(t) - \frac{N}{\tau_p^*}$$

$$\text{where } S_{gas}(t) = S_{gas,0} \exp\left(-\frac{t}{\tau_{gas}}\right); \tau_{gas} \sim 0.55 \text{sec}$$

$$N(t) - N_o = \eta_{NBI} S_{NBI} \tau_p^* - N_o \left[1 - \exp\left(-\frac{t}{\tau_p^*}\right) + \frac{\eta_{gas} S_{gas,0} \tau_{gas} \tau_p^*}{\tau_{gas} - \tau_p^*} \exp\left(-\frac{t}{\tau_{gas}}\right) - \exp\left(-\frac{t}{\tau_p^*}\right) \right]$$

Particle Containment Times ~ 0.5 sec. in NSTX with more realistic gas fueling rate model



**In all models, $\tau_p^* \sim 0.5 \text{ sec} \gg \tau_E$, i.e. $R_{\text{eff}} > 0.5$
 \gg NSTX is almost always high recycling with NBI**



- **Solutions to More Realistic Dependencies of Fueling Terms in Progress, i.e.**

$$\tau_p^*(t) = \tau_{p,0}^* (1 + \alpha t) \text{ (some L – modes with increasing confinement or recycling)}$$

$$S_{gas}(t) = S_{gas,0} (1 + \alpha t) \text{ (source increasing/decreasing with time)}$$

$$\eta_{gas}(t) = \eta_{gas,0} \exp -t / \tau_p^* \text{ (fuel efficiency decreasing with time)}$$

$$\eta_{gas}(t) = \eta_{gas,0} / N(t) \text{ (fuel efficiency decreasing with density)}$$

Where are particles and power in NSTX divertor?



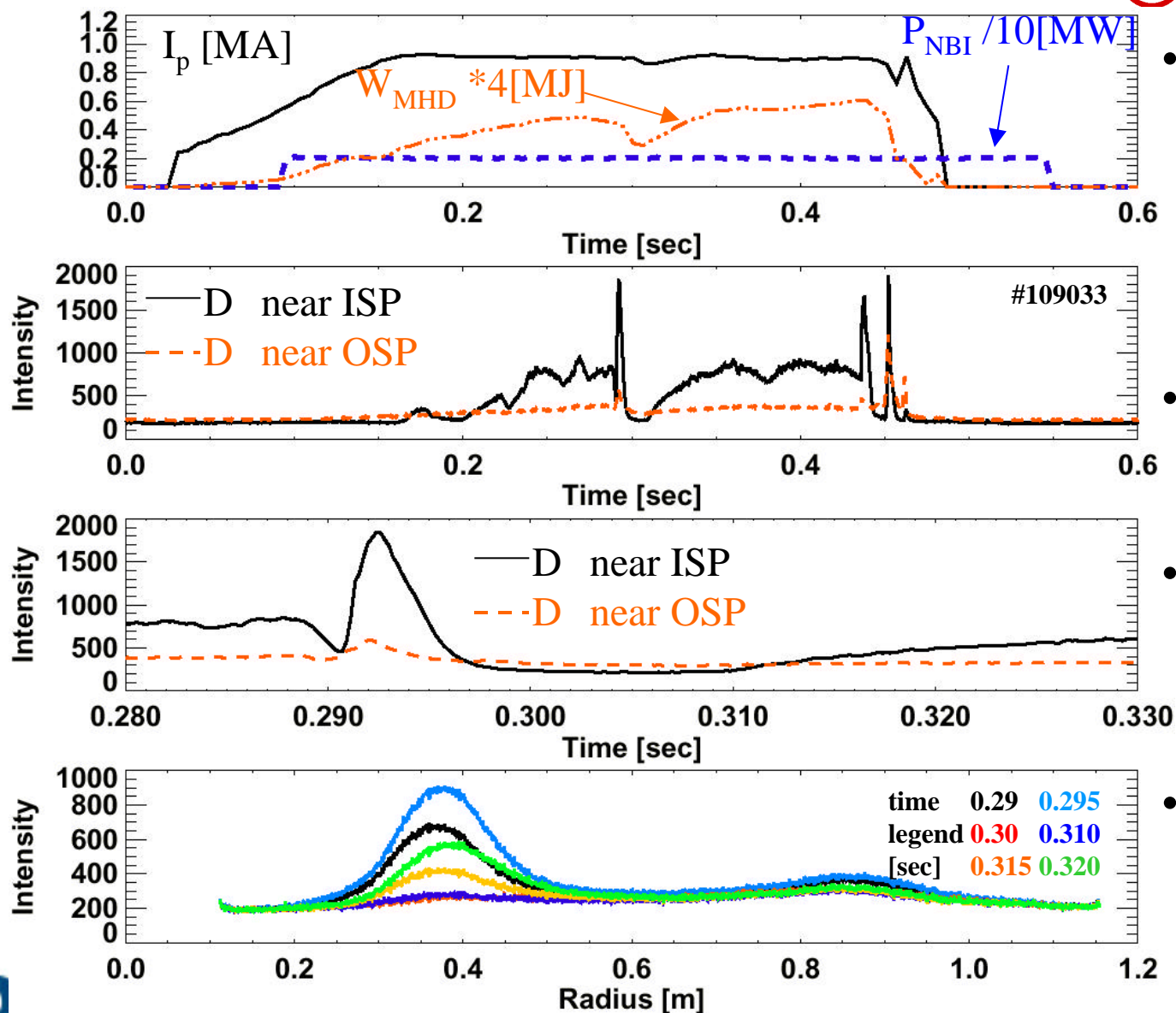
NSTX

- D peaks near inner and outer strike points, inner $\sim 3\times$ outer
 - Ratio reverses during power excursion \rightarrow inner probably detached
 - Most particles on outer side \rightarrow consistent with module location
- Heat flux always peaks near outer strike point
 - inner strike point peak heat flux and power $< 1/3$ outer values
 - $>$ consistent with module location

D_α profile normally peaks near Inner Strike Point, but reverses during heat pulse



NSTX

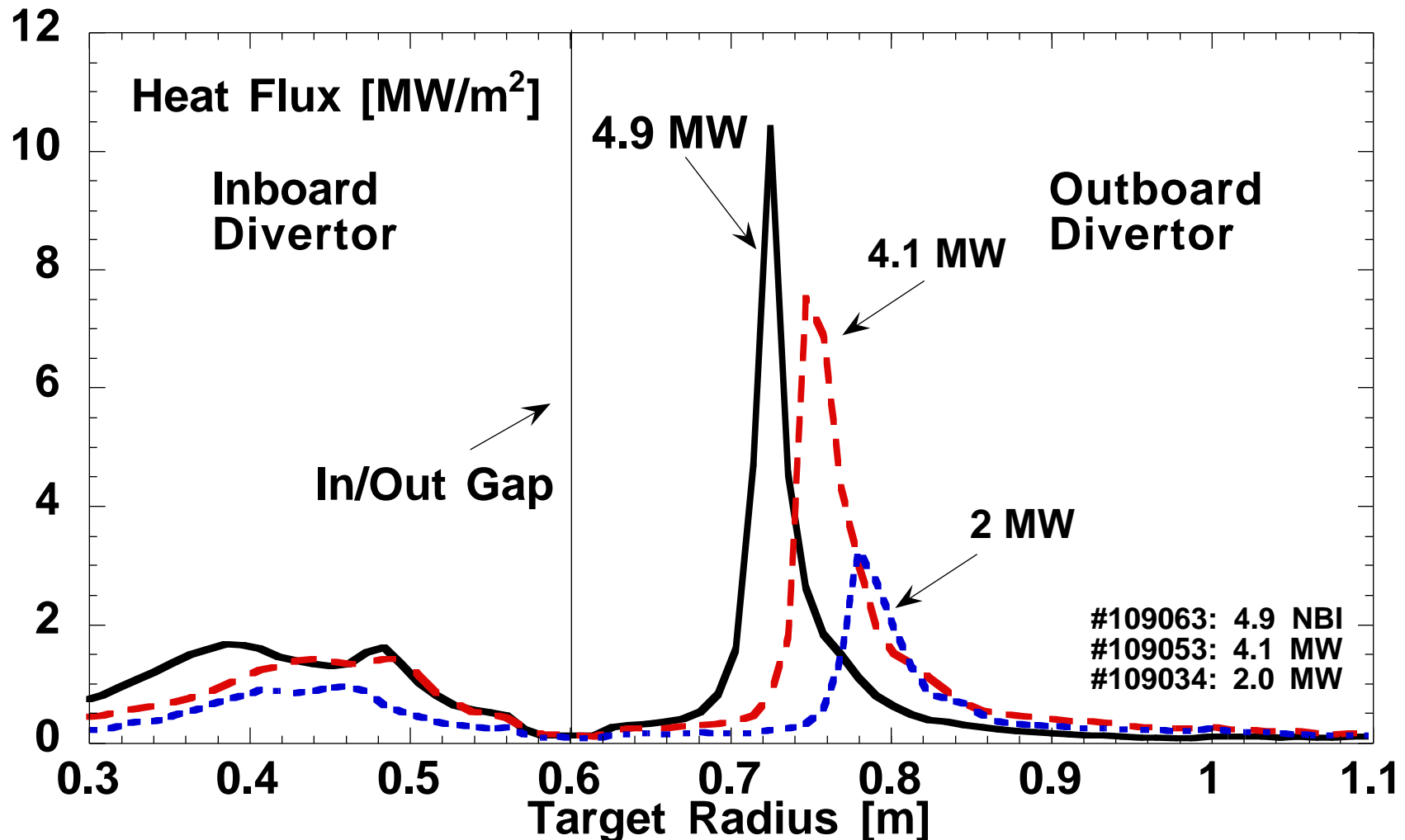


- Reversal of in/out D profile during heat pulse is consistent with inner side being detached
- Similar situation in pre-cryopump days at DIII-D
- Reduced recycling will probably re-attach ISP, so D will go down
- Likely more particles near OSP; need LP data to confirm

Peak heat flux always peaks near outer strike point in lower-single null configuration



- Good power accountability: $P_{\text{div}}^{\text{in+out}} \sim 70\%$ of P_{SOL}



Progress on Assessment of NSTX Divertor Particle and Heat Fluxes



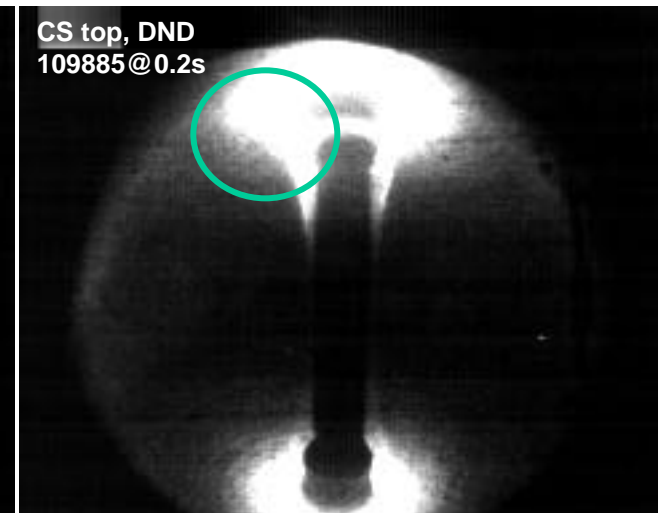
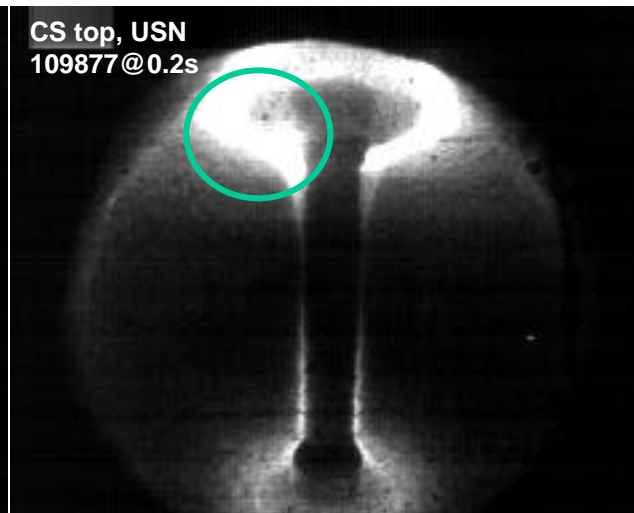
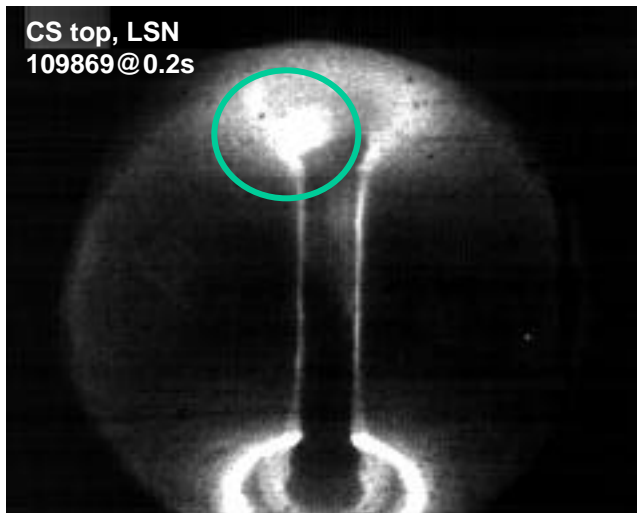
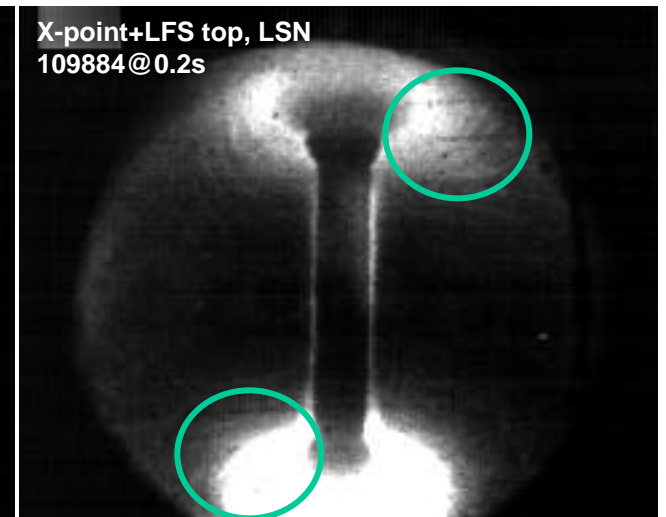
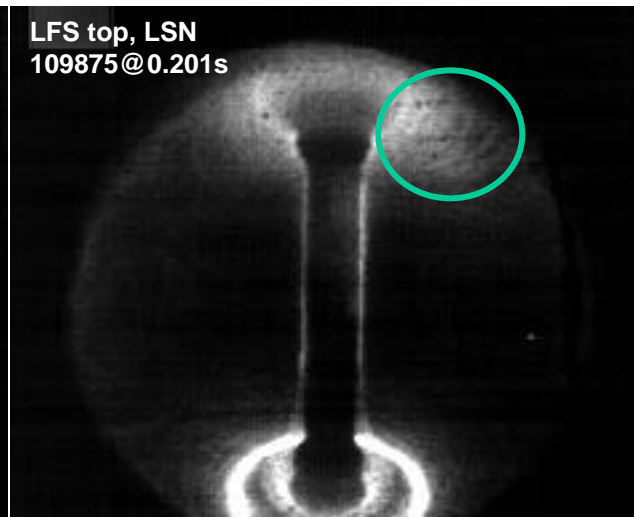
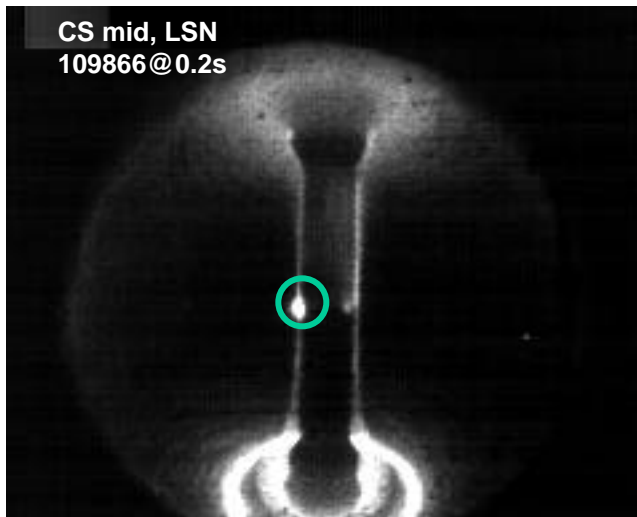
- NSTX is always (99%) in moderate-high recycling state and suffers from uncontrolled density rise
 - L-mode density increases with NBI fueling; no need for gas puffing
 - H-mode density increase > NBI fuel rate; $\tau_p^* \sim 0.2-0.5$ sec
- > Density control needed, but where are particles and power?
- D peaks near inner and outer strike points, inner $\sim 3\times$ outer
 - Ratio reverses during power excursion -> inner probably detached
 - Most particles on outer side -> consistent with module location
- Heat flux always peaks near outer strike point
 - inner strike point peak heat flux and power < 1/3 outer values
 - > consistent with module location
- UEDGE modeling in progress; DEGAS-2/TRANSP to follow to estimate effect of lower recycling

Backup: H-mode slides

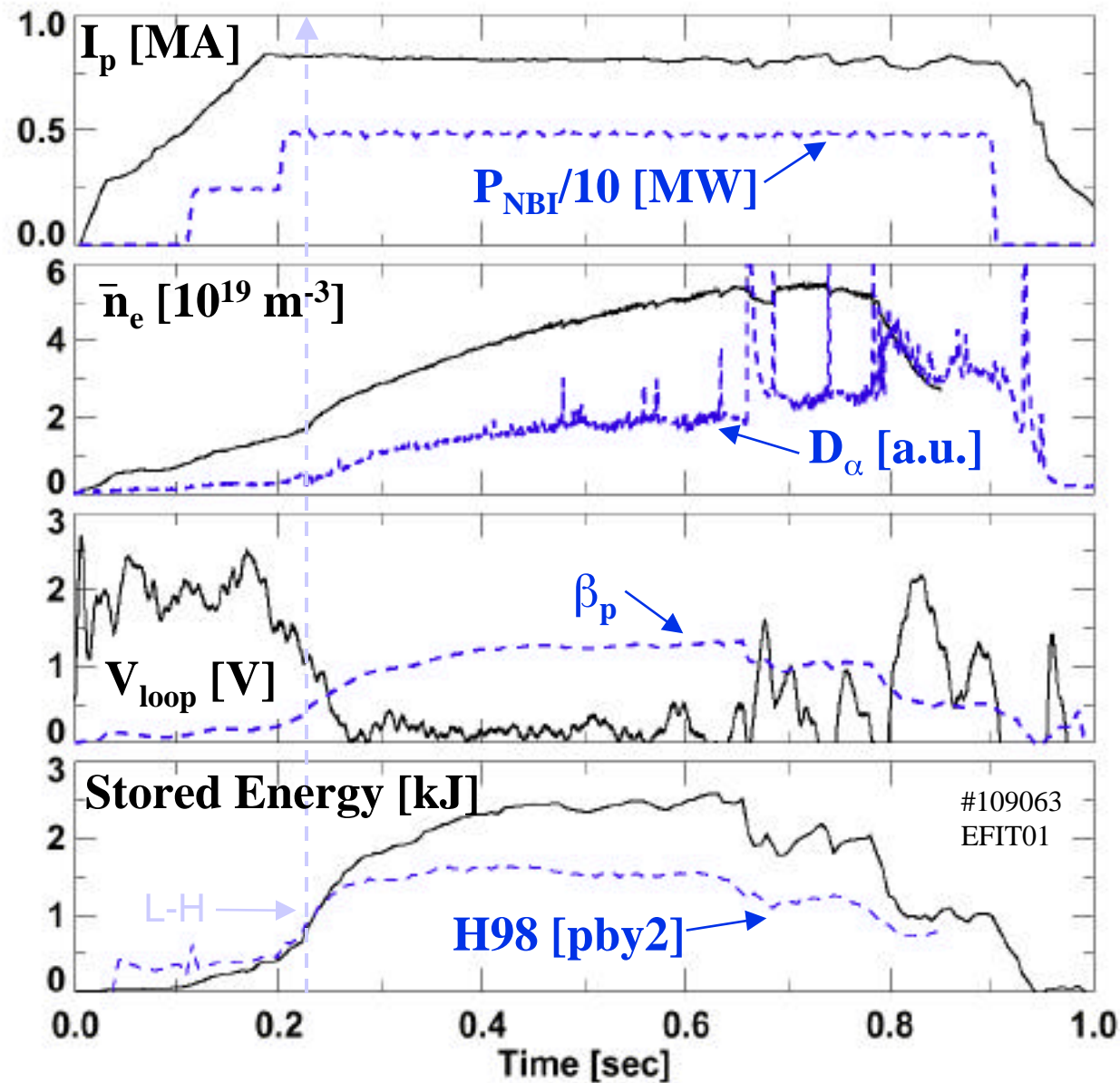


NSTX

Different Gas Puffers Light up Different Plasma Regions (unfiltered)



H-mode Plasmas Achieved Long Pulse, Owing to Low Volt-Second Consumption Rate



H-mode Plasmas Achieved High β_t , Owing to Reduced Pressure Peaking Factor

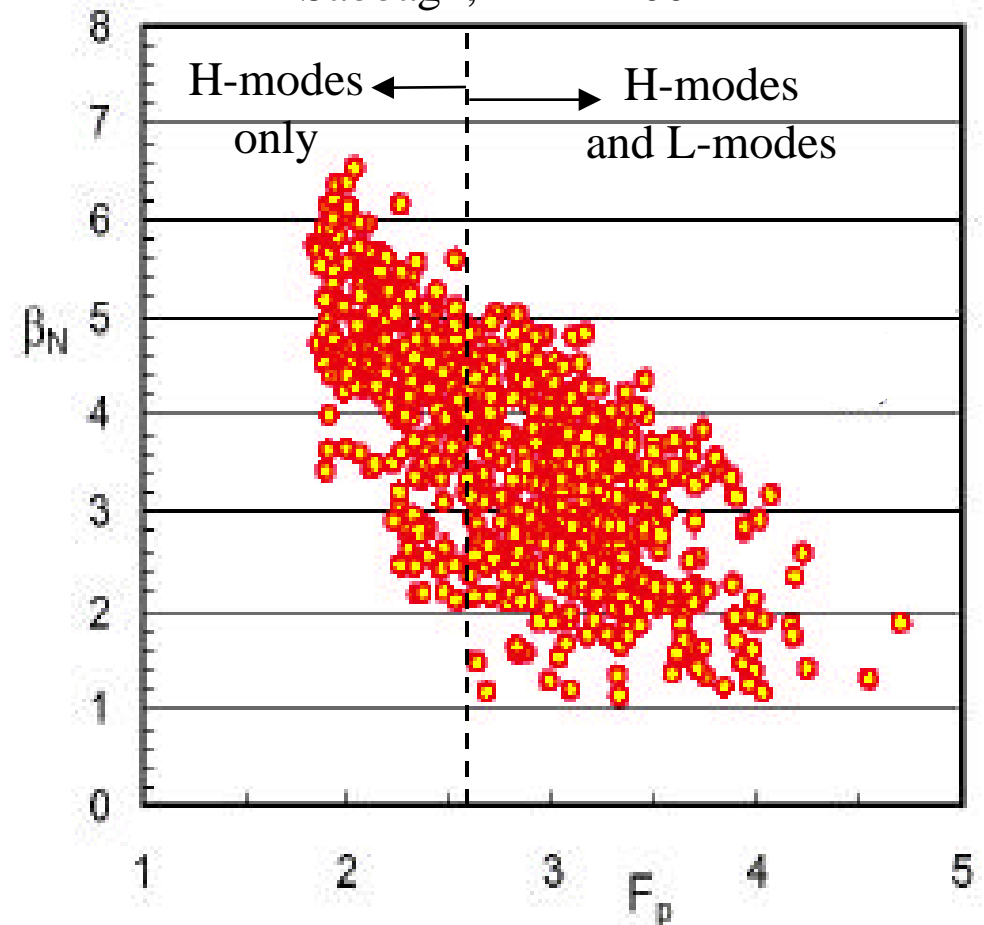
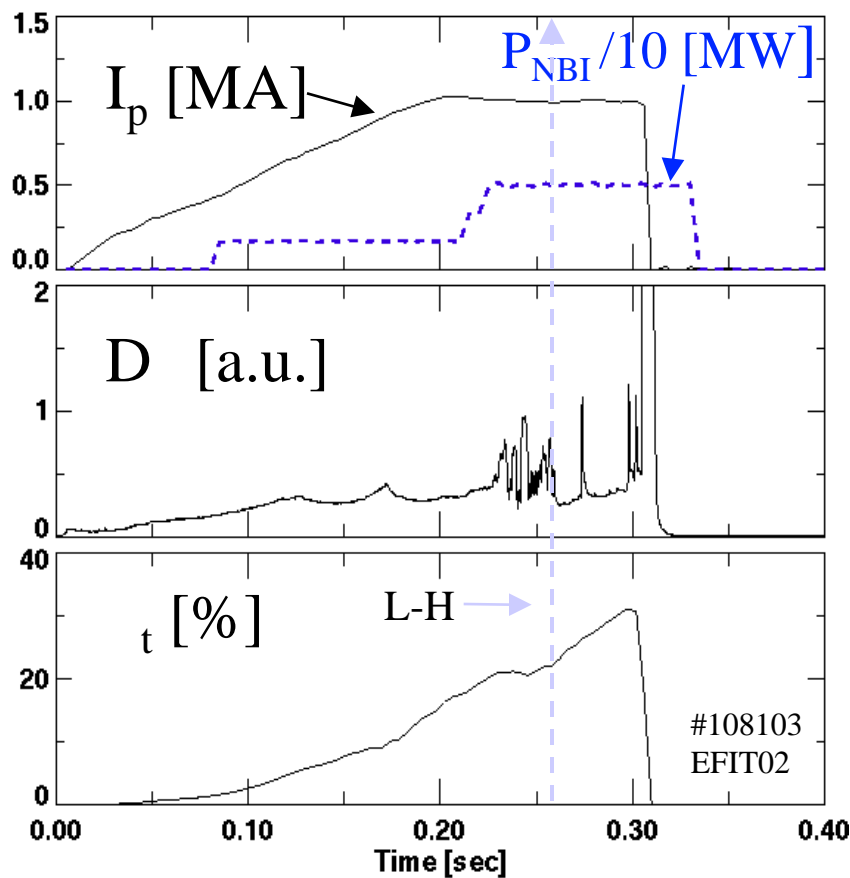


Max $\beta_T = 31.5\%$
 $\beta_N^{\max} \sim 6.2$

$$\beta_T = 2\mu_0 \langle p \rangle / B_0^2$$

Min $f_p(H) \sim 1.9$, Min $f_p(L) \sim 2.6$
 $f_p \equiv p_e / \langle p_e \rangle$

Sabbagh, IAEA 2002



L-H power threshold and ELM studies reveal differences with conventional aspect ratio tokamaks



- L-H transition: $P_{L-H}^{NSTX} > P_{th,1}$
- I_p dependence
- related to E_r through fast ion loss?

$$P_{th,1} \sim n_e^{0.61} B_T^{0.78} a^{0.89} R^{0.94}$$

